Facility Decommissioning Evaluation
Building 690-N, Process Heat Exchanger Repair Facility

G-FDE-N-00013

Revision 2

May 13, 2019

SAVANNAH RIVER SITE • AIKEN, SOUTH CAROLINA

Facility Decommissioning Evaluation
Building 690-N, Process Heat Exchanger Repair Facility

This is an Integrated Sampling Model Decommissioning per Facility Disposition Manual 1C

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Introduction

This document contains an evaluation of available existing information about a facility that is slated for decommissioning. This evaluation screens the project to determine whether it is appropriate to conduct the decommissioning under the Comprehensive Environmental Response, Compensation, and Liability Act (CERCLA), or to use a simpler graded approach.

This Facility Decommissioning Evaluation (FDE) consists of three sections. Part 1 contains a description of the project scope, including a brief summary of the purpose and history of the facility and photographs of the structures that are part of the project. Part 2 encompasses a series of questions, the answers to which determine the decommissioning model (CERCLA Model, Integrated Sampling Model, or Simple Model) that will be used. The three graded approach models are described in Facility Disposition Manual 1C, Procedure 501 (Reference 1). Part 2 also includes a justification for the answers to each question. Part 3 is a list of references that were used for the evaluation.

This revised FDE includes a summary of the 2014 enhanced characterization of Building 690-N as documented in the *Characterization Report Building 690-N (Ford Building) and Ancillary Equipment and Facilities* (Reference 10) and the *Human Health Risk Screening Evaluation for the 690-N Ford Building* (Reference 12). Sampling results from the 2014 characterization report and the Part 2 Evaluation checklist support the Integrated Sampling Model for decommissioning of Building 690-N.

Conclusion

This facility is contaminated, and the facility hazard category is Other Industrial. A review of the existing characterization data, process/building history, sample data and walk downs of the facility, supports the determination that this building and its ancillary structure meet the criteria for an Integrated Sampling Model decommissioning as described in Facility Disposition Manual 1C, Procedure 501. This project will be conducted using the Integrated Sampling Model. Post-decommissioning facility remnants will be closed under CERCLA in conjunction with the ECODS N-1, Central Shops Scrap Lumber Pile (631-2G) and Building 690-N, Process Heat Exchanger Repair Facility (Ford Building) Operable Unit (OU). Prior to demolition of 690-N, a Post-Decommissioning Action Plan will be prepared and issued, addressing Surveillance & Maintenance (S&M) and any continuing monitoring requirements following decommissioning. Responsible personnel, costs, and frequency of S&M/monitoring activities will be addressed in the Post-Decommissioning Action Plan, to the extent possible.

Part 1. Project Scope

Scope

This Evaluation has been prepared in accordance with requirements found in Facility Disposition Manual 1C, Procedure 502, "Preparing Decommissioning Decision Documents" (Reference 2). The original (Revision 0 and 1) scope of this evaluation included the following building and ancillary structures:

- Building 690-N, Process Heat Exchanger Repair Facility.
- 13.8 kV Substation 652-44N, remnants.
- Excess Equipment Yard 745-N, including contents.
- Fuel Oil Tank Concrete Containment Dike.

As a function of the approved Fiscal year 2014 Federal Facility Agreement (FFA) Appendix E, which included separation of some waste units from their respective Area Operable Units (OU) to meet expectations for cleanup acceleration provided by SCDHEC and EPA in the approval of the FY2013 Appendix E, Savannah River Site (SRS) will address the 13.8 kV Substation Remnants (652-44N) as a component of the ECODS N-1, Central Shops Scrap Lumber Pile (631-2G) and Building 690-N, Process Heat Exchanger Repair Facility (Ford Building) Operable Unit (OU). The ECODS N-1, Central Shops Scrap Lumber Pile (631-2G) and Building 690-N, Process Heat Exchanger Repair Facility (Ford Building) Operable Unit (OU) are scheduled for a characterization field start in FY2020. The field start will be preceded by pre-workplan characterization initiated in FY2019.

As indicated by the enhanced characterization results as documented in the *Characterization Report Building 690-N (Ford Building) and Ancillary Equipment and Facilities*, SDD-2014-00054, Rev. 1, August 18, 2014, and the *Human Health Risk Screening Evaluation for the 690-N Ford Building,* SDD-2019-00030, Rev. 0, April 11, 2019, the majority of the facility does not represent a contamination concern; however, PCBs are present in the concrete floor slab at levels that warrant further evaluation. Based on the FDE Part 2 Evaluation checklist and the sample data in the 2014 characterization report, SRS believes that the Integrated Sampling Model is the appropriate model for decommissioning of Building 690-N, excluding the slab. SRS proposes to proceed with the decommissioning of Building 690-N by installing a protective cover over the slab prior to demolition, which will prohibit disturbance and potential spread of PCB contamination during demolition and removal activities. Post-decommissioning facility remnants (including the building slab) will be closed under the FFA in conjunction with the ECODS N-1, Central Shops Scrap Lumber Pile (631-2G) and Building 690-N, Process Heat Exchanger Repair Facility (Ford Building) Operable Unit (OU). Prior to demolition of 690-N, a Post-Decommissioning Action Plan will be prepared and issued, addressing Surveillance & Maintenance (S&M) and any continuing monitoring requirements following decommissioning. Responsible personnel, costs, and frequency of S&M/monitoring activities will be addressed in the Post-Decommissioning Action Plan, to the extent possible. This approach will physically and administratively separate the slab from the rest of the structure allowing SRS to take advantage of increased near-term program funding to remove the structure while providing the opportunity to

evaluate alternatives for final disposition of the building remnants. The Facility Decommissioning Evaluation Building 690-N, Process Heat Exchanger Repair Facility, Revision 1 document is being revised herein as an Integrated Sampling Model and to include a description of the enhanced characterization and summarize the results that demonstrate there is no contamination concern, other than what is associated with the slab.

The 745-N Excess Equipment Yard, including the 49 heat exchangers and Deionizer Trailer, are no longer part of the Building 690-N FDE scope. In 2011 and 2012, the 49 heat exchangers were characterized and disposed of in the SRS Solid Waste Burial Ground (E-Area) as part of an American Recovery and Reinvestment Act (ARRA) deactivation project. During the loading of the 49 heat exchangers for disposal, the concrete slab cracked under the combined weight of the large crane and heat exchangers. The 745-N concrete slab was then sampled and found to have some slight surface contamination due to condensation leakage from heat exchangers (particularly Exchanger #3). The slab was removed to eliminate any risk of residual contamination reaching the soil, and the soil sampled and found to be non-contaminated with respect to the need for any SRS radiological controls or postings. Results of the soil sampling are documented in the *745-N Survey for Rollback of CA,* RSLS-DDSG-M-20150713-1 (Reference 8). Under slab piping (i.e., sanitary sewer, domestic water, and fire water) was also removed and the area covered with top soil and seeded.

The Deionizer Trailer is a portable equipment item that was incorrectly included in the scope of the previous 690-N FDE and is likewise outside the scope of Revision 2 of the 690-N FDE. The Deionizer Trailer will be dispositioned as a piece of equipment via deactivation activities in accordance with SRS procedures and state and federal regulations.

The proposed decommissioning end-state for Building 690-N is demolition to the building concrete slab. Prior to building demolition, SRS proposes to install a protective cover over the building slab, which will prohibit disturbance and potential spread of PCB contamination during demolition and removal activities. Following demolition, SRS proposes to install a 6-inch thick reinforced concrete cap slab over the building footprint. The decommissioning end-state for the Fuel Oil Tank Concrete Containment Dike is partial removal (i.e., cleaving) of its concrete wall to allow drainage of rainwater.

As indicated by the subsequently performed enhanced characterization results provided within the *Characterization Report Building 690-N (Ford Building) and Ancillary Equipment and Facilities*, SDD-2014-00054, Rev. 1, August, 18, 2014, and the *Human Health Risk Screening Evaluation for the 690-N Ford Building,* SDD-2019-00030, Rev. 0, April 11, 2019, minimal radiological, PCB, or RCRA hazardous contamination is present in the concrete floor slab, soils below the concrete floor slab, steel hut containment walls, and the ancillary equipment and facility. Based on the FDE Part 2 Evaluation checklist and the sample data in the 2014 characterization report, the Integrated Sampling Model is the appropriate model for decommissioning of 690-N and its ancillary equipment and facility.

Contaminants and/or releases to the environment, not associated with the facility being decommissioned, are not within the scope of the decommissioning project.

The described decommissioning activities are not the final area closure actions. The decommissioning of a building is intended to reduce landlord costs, increase safety by removing excess facilities and reduce the potential for releases of hazardous substance to the environment.

Facility Description

Building 690-N, Process Heat Exchanger Repair Facility is classified as an "Other Industrial" facility in the Savannah River Nuclear Solutions "Standards/Requirements Identification Document Facility List" (see Reference 6). It was used primarily as a machining repair/rework shop for contaminated SRS reactor process water (i.e. Deuterium Oxide) heat exchangers.

Building 690-N, Process Heat Exchanger Repair Facility, built in the 1950s, is a one-story metal frame structure on a concrete slab covering approximately 9,700 square feet. It is located in the extreme southeastern tip of the Central Shops (N-Area) near the center of SRS. The primary area consists of a machine shop with offices, storage rooms, restrooms and service area, which can only be accessed from the outside of the building on the north side. In the mid-1960s, a sealed shell was installed inside the original building frame with a ventilation and High Efficiency Particulate Air (HEPA) filter system to maintain a slight negative pressure and allow the facility to serve as a repair shop for leaking SRS reactor contaminated process water heat exchangers. The HEPA equipment was removed in 2012 as part of an American Recovery and Reinvestment Act (ARRA) deactivation project." The ground in the area was covered with plastic and huts erected to contain any contamination. HEPA Housings, filters, and ductwork were loaded into Sealand containers and disposed of in the SRS Solid Waste Burial Ground (E-Area). Following removal of the HEPA Housings, the curbed concrete containment pads (i.e., sumps) beneath them were sampled and found to be slightly contaminated. "The curbed containment pads and sumps were filled with concrete and each capped with a reinforced concrete slab. Radiological monitoring and sampling throughout the removal iteration showed non-detect readings except on the concrete of the curbed sumps which are now filled in and covered with concrete.

The shop area also contains a single 1'-6" X 1'-6" X 1'-0" sump located near the east end of the shop area, between the two sets of railroad tracks, at the radial drill.

Prior to installation of the inner shell and HEPA Filter Systems, 690-N was a "clean" facility, ventilated in Summer months by eave fans E.P. 20901-A (East End) and E.P. 20901-B (West End). Exhaust fans were blocked off during cold months of the year. Building modifications to change 690-N's mission to a heat exchanger maintenance repair/rework facility from a "clean" facility installed the inner building containment shell and HEPA Filter Systems to preclude radiological contamination of the clean portions of the building and environmental media outside the facility. When the building was modified for its heat exchanger maintenance repair mission, the West eave fan was relocated to the new West wall. Both fans continued to ventilate hot air from the attic and other clean portions of the main building until the building was shut down.

This facility was used during the 1950s exclusively for testing of Ford Company-manufactured motor control packages for control rod drive mechanisms prior to their installation in the SRS reactors. During the early 1960s, the SRS reactors were operating at higher power levels and failure (i.e., leakage) of heat exchangers prompted conversion of this facility for heat exchanger maintenance repair/rework. This mission continued until the early 1980s when the procurement of new heat exchangers for the SRS reactors terminated the maintenance repair mission of the facility. In the early through mid-1980s, it housed Construction crews that performed final assembly and testing of new, clean heat exchangers. For the remainder of the 1980s, the building was used to store equipment and miscellaneous supplies. During the early 1990s, the K-Reactor had a minor leak in a heat exchanger that resulted in reactivating the facility. The facility operated again as a heat exchanger repair facility for about six months and was subsequently closed. It was then utilized to store excess equipment (in waste containers [e.g. Sealands] and/or bagged/wrapped in plastic) that was chemically and/or radiologically contaminated. Cleanup activities that began in 1998 prompted the removal of stored excess equipment (see Reference 7).

The following ancillary facility is associated with 690-N:

1. The concrete containment dike, located to the south of Building 690-N, contained a fuel oil tank that has been removed. The fuel oil tank was used to supply the 690-N Building heating systems.

Services and utilities to and or from the facility included plant air, fuel oil, domestic water, fire water, electrical power, sanitary sewer and process sewer. All services to 690-N and its ancillary facility have been isolated and air-gapped, rendering the facilities "cold and dark."

Figures 1, 2 and 3 depict the Building 690-N Heat Exchanger Repair Facility, its location, and its floor plan. Figure 4 depicts the ancillary structure (Fuel Oil Tank Concrete Containment Dike).

Figure 1 – Building 690-N, Process Heat Exchanger Repair Facility (Looking Southeast)

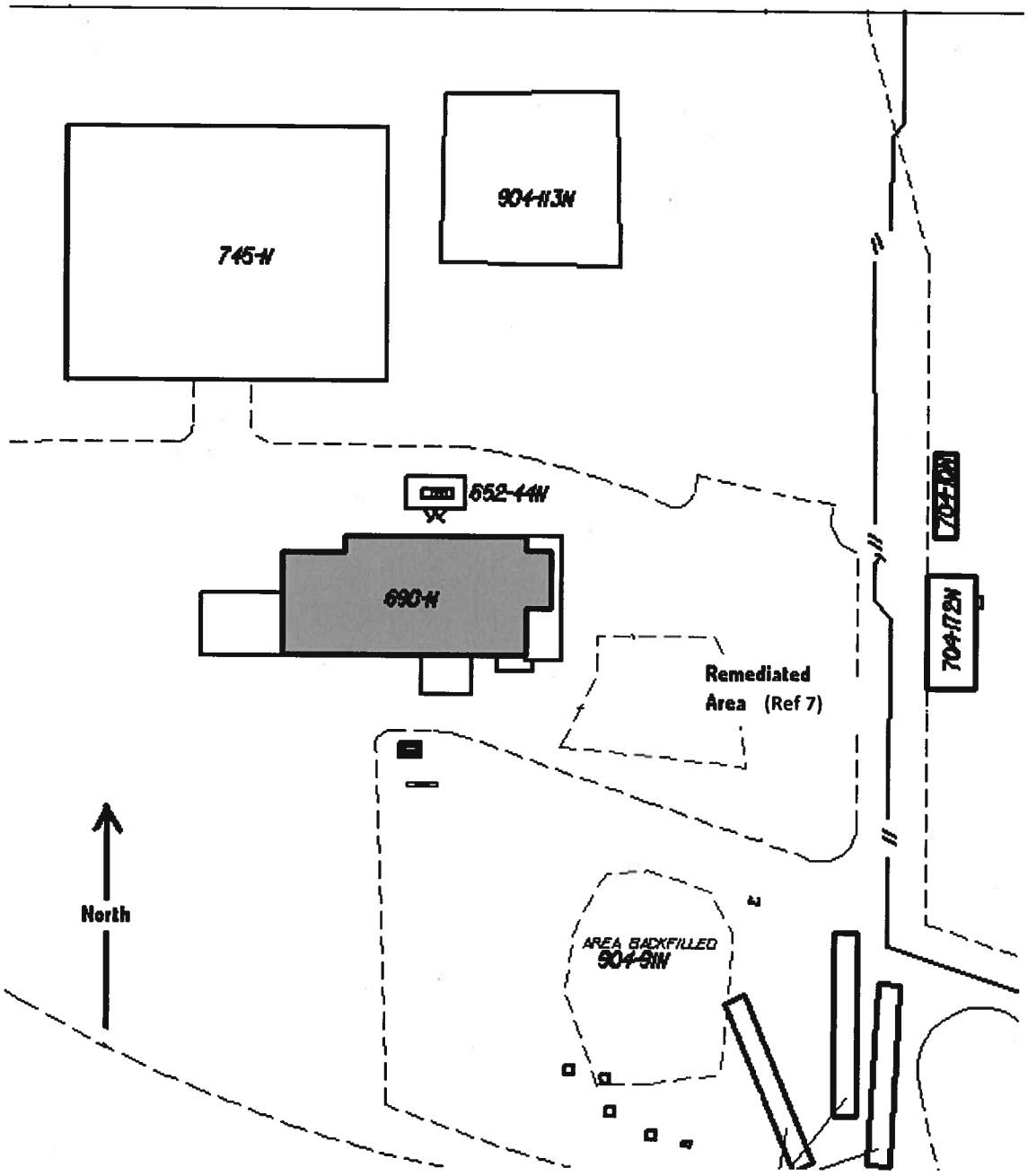
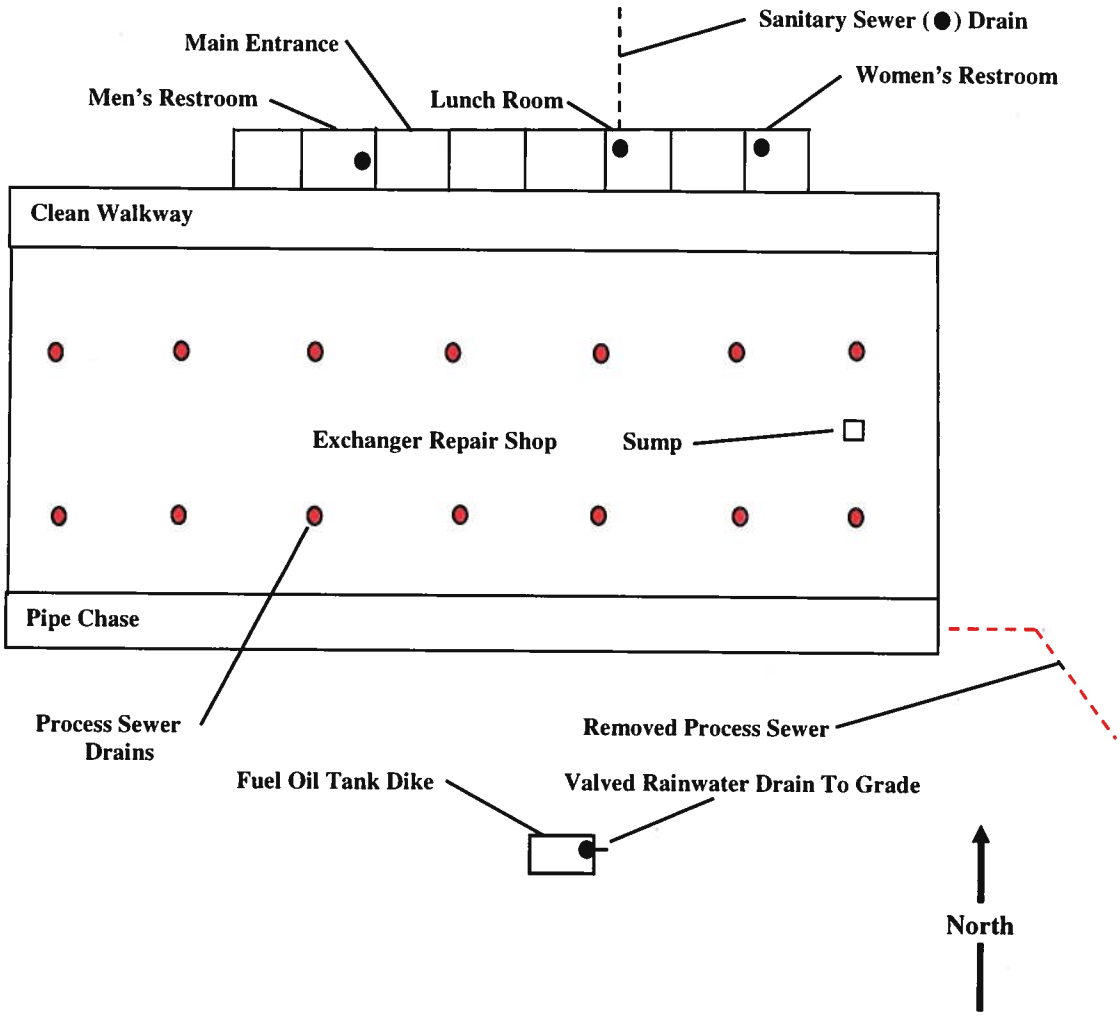


Figure 2 – Building 690-N, Process Heat Exchanger Repair Facility (Location)[1]

[1] Building 690-N is located in the southeast corner of N-Area and approximately 1,000' south of Road 3.

Not To Scale



(See Reference 7 for previous cleanup effort involving the removed process sewer.)

Figure 3 – Building 690-N, Process Heat Exchanger Repair Facility (Floor Plan)

Figure 4 -Fuel Oil Tank Concrete Containment Dike (Looking East)

Process History

Review of records, walk downs and interviews indicate that no chemical or radioactive processes were performed in this building (i.e., no chemical, mechanical, or electrical energy or interaction was performed to change the state of an input material or to produce a new output product).

Initially, the building was used for testing of new, clean equipment prior to installation of that equipment into the reactor facilities. The building was subsequently used for maintenance repairs to reactor process water heat exchangers. This was followed by assembly and testing of new, clean reactor process water heat exchangers. Finally, the building was used for storage of contaminated equipment and materials, with only a single period of about six months during the 1990s when it was again used for maintenance repairs to a single K-Reactor heat exchanger.

Chemical Process

Chemical Name	*Process location*	*Evidence of spills?*
Not Applicable	Not Applicable	Not Applicable

Radioactive Process

Isotope	*Contaminated areas/others*
Not Applicable	Not Applicable

There are posted Inactive Contamination Areas (CAs) in the facility, with the shop area being the largest Inactive CA. A full characterization of 690-N has been performed and is documented in SDD-2014-00054, Rev. 1 (Reference 10).

As indicated by the subsequently performed enhanced characterization results provided within the *Characterization Report Building 690-N (Ford Building) and Ancillary Equipment and Facilities*, SDD-2014-00054, Rev. 1, August, 18, 2014, and the *Human Health Risk Screening Evaluation for the 690-N Ford Building,* SDD-2019-00030, Rev. 0, April 11, 2019, minimal radiological, PCB, or RCRA hazardous contamination is present in the concrete floor slab, soils below the concrete floor slab, steel hut containment walls, and the ancillary equipment and facility. Based on the FDE Part 2 Evaluation checklist and the sample data in the 2014 characterization report, the Integrated Sampling Model is the appropriate model for decommissioning of the 690-N and its ancillary equipment and facility.

The heat exchanger maintenance repair work generated wastewater contaminated with low levels of radioactivity and trace quantities of non-radioactive organic and inorganic compounds. Repair workers sent the wastewater to a 6,000-gallon underground retention tank adjacent to the Ford Building and the wastewater was analyzed for radionuclides. Depending on the results, the wastewater was either released to the Ford Building Seepage Basin through an underground

process sewer pipeline or transferred to other SRS operations for proper disposal. In 1998, a project was undertaken by SRS to remediate the associated retention tank, pumping station, process sewer lines and seepage basin of the Ford Building (see Reference 7). These remediated equipment/components are not within the scope of this FDE. Building 690-N has fourteen (14) process sewer drains in the shop area that have been plugged with concrete. When the building was modified for heat exchanger maintenance, the floor drain piping was installed on top of the 4" slab and a second 8" thick concrete floor slab was poured atop the 4" slab, encapsulating the process sewer lines within the 8" slab. Piping from the plugged floor drains to the perimeter of the building slab, near its southeast corner, remains sandwiched between the building's double floor slab. Piping beyond this point was removed by the 1998 project.

The shop area also contains a single 1'-6" X 1'-6" X 1'-0" sump located near the east end of the shop area, between the two sets of railroad tracks, at the radial drill.

The sanitary system has been isolated, but fixtures are still installed.

A visual inspection of the building floor was performed and there were no visible cracks that appeared to penetrate the slab. Note that the heat exchanger repair shop has several epoxy coatings applied to the floor sealing residual PCB and radioactive contamination. These coatings may hide cracks in the concrete floor.

A visual inspection of the tank containment dike was performed and there were no visible cracks that appear to penetrate the slab. There is no evidence of spills (e.g., oil stains); however, dirt and water stains are present. There were no visible stains around the tank containment perimeter.

Summary of Existing Characterization

Characterization has been accomplished using a combination of process knowledge/historical release information, verification walk downs, and sampling, as appropriate.

Contaminants and/or releases to the environment that are not associated with the facility(s) being decommissioned, are not within the scope of the decommissioning project.

An important part of the characterization portion of this evaluation is a historical review of spills/releases to the environment. This review includes a review of the Occurrence Reporting and Processing System/Site Item Reportability and Issue Management (ORPS/SIRIM) database (Reference 11) conducted from the effective date of the Federal Facility Agreement (FFA), August 16, 1993, to present and a review of the FFA (Reference 4). The FFA serves as a review of releases/spills to the environment prior to August 16, 1993. Neither the ORPS/SIRIM database nor the FFA listed a release/spill for Building 690-N. As indicated by the subsequently performed enhanced characterization results provided within the *Characterization Report Building 690-N (Ford Building) and Ancillary Equipment and Facilities*, SDD-2014-00054, Rev. 1, August, 18, 2014, and the *Human Health Risk Screening Evaluation for the 690-N Ford Building,* SDD-2019-00030, Rev. 0, April 11, 2019, minimal radiological, PCB, or RCRA hazardous contamination is present in the concrete floor slab, soils below the concrete floor slab, steel hut containment walls, and the ancillary equipment and facility. Based on the FDE Part 2

Evaluation checklist and the sample data in the 2014 characterization report, the Integrated Sampling Model is the appropriate model for decommissioning of the 690-N and its ancillary equipment and facility.

In October 1997, it was determined that cutting oil used in some of the machinery inside radiological-contaminated areas contained Polychlorinated Biphenyls (PCB's), a Toxic Substances Control Act (TSCA) regulated chemical. To address spills within the building associated with the cutting oil, SRS conducted cleanup activities beginning in 1998 under the provisions of 40 CFR 761 Subpart D, specifically 40 CFR 761.61 (a) (see Reference 3). The cleanup activities included decontamination along with initial and subsequent application of a floor coating. Subsequently, SRS performed a further full characterization of the facility using more aggressive methods of characterization as documented in the *Characterization Report Building 690-N (Ford Building) and Ancillary Equipment and Facilities*, SDD-2014-00054, Rev. 1, August 18, 2014 (Reference 10) and the *Human Health Risk Screening Evaluation for the 690-N Ford Building,* SDD-2019-00030, Rev. 0, April 11, 2019. Soil PCB sample results show that no samples exceeded the TSCA threshold of 50 ppm. In addition, the 2014 characterization results (Reference 10) indicate only minimal radiological or RCRA hazardous contamination is present in the floor slab, soils below the slab, and the steel hut containment walls of the 690-N facility or ancillary structure.

Wastes generated during decommissioning will be characterized and managed in accordance with SRS procedures and State and Federal regulations.

Historical Significance

A review has been conducted in accordance with a Programmatic Agreement. This review resulted in the publication of a Cultural Resources Management Plan (Reference 5), which lists the facilities of historical significance at SRS. This facility is not listed in that reference and therefore is not historically significant.

Part 2. Evaluation

Clean Facilities

	Question	Yes	No	Justification
1.	Has the facility ever contained or processed radioactive or hazardous material other than stored packaged material or materials of construction? *If yes, go to question 4.*	X		This facility was used during the 1950s exclusively for testing of Ford-Company-manufactured motor control packages for control rod drive mechanisms prior to their installation in the SRS reactors. During the early 1960s, the SRS reactors were operating at higher power levels and failure (i.e., leakage) of heat exchangers prompted conversion of this facility for heat exchanger maintenance repair/rework. This mission continued until the early 1980s when the procurement of new heat exchangers for the SRS reactors terminated the maintenance repair mission of the facility. In the early to mid-1980s, it housed Construction crews that performed final assembly and testing of new heat exchangers. For the remainder of the 1980s, the building was used to store equipment and miscellaneous supplies. During the early 1990s, the K-Reactor had a minor leak in a heat exchanger that resulted in reactivating the facility. The facility operated for about six months and was subsequently closed. It was then utilized to store excess equipment (in waste containers (e.g. Sealands) and/or bagged/wrapped in plastic) that was chemically and/or radiologically contaminated. Cleanup activities that began in 1998 prompted the removal of stored excess equipment (see Reference 7). Machining equipment used for maintenance repairs of heat exchangers contained PCB-contaminated oils that leaked onto the exchanger repair shop area floor (see Reference 3).
2.	If there was stored packaged material, has there ever been a spill? *If No or N/A, this is a Simple Model. Stop.*			N/A (Question was skipped as directed by Question 1.)
3.	Was spill confined inside structure and cleaned to free release standard per Radiological Control Manual 5Q (for radiological) or continued occupancy per Industrial Hygiene Manual 4Q (for hazardous)? *If Yes, this is a Simple Model. Stop.*			N/A (Question was skipped as directed by Question 1.)

Contaminated Facilities

	Question	Yes	No	Justification
4.	Is the facility listed as a Resources Conservation and Recovery Act (RCRA)/CERCLA Unit in Appendix C of the SRS FFA? *If Yes, this is a CERCLA Model. Stop.*		X	The facility is not listed as a RCRA/CERCLA Unit in Appendix C of the SRS FFA (see Reference 4).
5.	Is the Facility listed as a Site Evaluation Area in Appendix G of the SRS FFA? *If Yes, this is a CERCLA Model. Stop.*		X	The facility is not listed as a Site Evaluation Area in Appendix G of the SRS FFA (see Reference 4).

	Question	Yes	No	Justification
6.	Is there evidence that there has been a release of hazardous or radioactive materials outside the structure? *If Yes, this is a CERCLA Model. Stop.*		X	A historical review of spills/releases to the environment, which included a review of the Occurrence Reporting and Processing System/ Site Item Reportability and Issue Management (ORPS/SIRIM) database (Reference 11) conducted from the effective date of the Federal Facility Agreement (FFA), August 16, 1993, to present and a review of the FFA (Reference 4), which serves as a review of releases/spills to the environment prior to August 16, 1993, showed no record of any spills from 690-N to the external environment.
7.	Is there a substantial threat of a release of hazardous or radioactive materials outside the structure? *If Yes, this is a CERCLA Model. Stop*		X	While some slight contamination is present inside the heat exchanger maintenance repair area, the contamination is "fixed" and poses no threat, substantial or otherwise, of a release of hazardous or radioactive materials outside the structure, as supported by SDD-2014-00054, Rev. 1, "Characterization Report Building 690-N (Ford Building) and Ancillary Equipment and Facilities", dated August 18, 2014 (Reference 10) and, SDD-2019-00030, Rev. 0, "Human Health Risk Screening Evaluation for the 690-N Ford Building", dated April 11, 2019 (Reference 12)
8.	Has the facility been assigned a hazard category as defined in Facility Safety Document Manual 11Q? *If No, stop and refer facility for evaluation to assign a hazard category, then proceed.*	X		Building 690-N is listed in the Standards/Requirements Identification Document (see Reference 6) as "Other Industrial".
9.	Is the hazard category Nuclear (HC- 2 or 3), Radiological, or High Hazard Chemical? *If Yes, this is a CERCLA Model. Stop.*		X	The 690-N structure has never been a Nuclear (HC-2 or HC-3), Radiological, or High Hazard Chemical facility.
10.	Has DOE-SR directed that the decommissioning be performed using the CERCLA Model? *If yes, this is a CERCLA Model. Stop.*		X	DOE-SR has not directed the use of the CERCLA Model to decommission Building 690-N.
11.	Does the complexity of the facility or the nature and extent of contamination warrant a higher than normal level of rigor and detail for decommissioning planning and evaluation? *If Yes, this is a CERCLA Model. Stop.*		X	The facility is neither complex nor does the level of contamination warrant a higher than normal level of rigor for its decommissioning.
12.	Is the facility a formerly nuclear, radiological, or high-hazard chemical facility? *If Yes, this is an Integrated Sampling Model. Stop.*		X	The 690-N structure has never been a Nuclear (HC-2 or HC-3), Radiological, or High-Hazard Chemical facility nor has it had its hazard category reduced for any reason.
13.	Has EC&ACP's Regulatory Support Group determined that a final survey is not required for this facilty? *If Yes, this is a Simple Model. If No, this is an Integrated Sampling Model. Stop*		X	The 690-N slab is the only component of the facility that remains a concern due to presence of PCB's. SRS commits to place slab and remnants of 690-N into appropriate ECODS N-1, Central Shops Scrap Lumber Pile (631-2G) and Building 690-N, Process Heat Exchanger Repair Facility (Ford Building) Operable Unit (OU). CERCLA evaluation and remedial process.

Part 3. Review of Existing Records

The following facility records were reviewed as a part of this evaluation:

Ref #	Document No.	Revision/Date	Title
1	Manual 1C, Procedure 501	Rev. 5, 12/31/2014	"Decommissioning of Facilities"
2	Manual 1C, Procedure 502	Rev. 4, 12/31/2014	"Preparing Decommissioning Decision Documents"
3	ESH-98-0340	N/A November 30,1998	Notification of Polychlorinated Biphenyls (PCB) Cleanup Activities at the Savannah River Site "Ford Building".
4	WSRC-OS-94-42	N/A / August 16, 1993 This item/reference includes approved Modifications to Appendix C (05/12/08) and G (05/12/08).	Federal Facility Agreement for the Savannah River Site, Administrative Document Number 89-05-FF, Effective Date: August 16, 1993.
5	None	Final/January 26, 2005	Savannah River Site's Cold War Built Environment Cultural Resources Management Plan, Volume One.
6	SRNS-RP-2008-00086-000-M&O	18-01-MO / February 15, 2018	Standards/Requirements Identification Document, Functional Area 00, Facility List.
7	WSRC-RP-2003-4038	Revision 1 / September 2003.	Post-Construction Report (PCR) / Corrective Measures Implementation Report (CMIR) / Final Remediation Report (FFR) for the Ford Building Seepage Basin (904-91G).
8	RSLS-DDSG-M-20150713-1	N/A / 8/28/2015	745-N Survey for Rollback of CA
9	RSLS-DDSG-M-20150714-1	N/A / 9/23/2015	745-N Soil Sampling for Rollback of CA
10	SDD-2014-00054	Rev. 1, August 18, 2014	Characterization Report Building 690-N (Ford Building) and Ancillary Equipment and Facilities
11	N/A	N/A	SRS Site Item Reportability Issue Manual (SIRIM)/Occurrence Reporting and Processing System Information System (ORPS) 8/16/93 to Present
12	SDD-2019-00030	Rev. 0, April 11, 2019	Human Health Risk Screening Evaluation for the 690-N Ford Building (U)